WASTE STREAM 9B310 Stainless Steel (Reactor) ILW

SITE Bradwell

SITE OWNER Nuclear Decommissioning Authority

WASTE CUSTODIAN Magnox Limited

WASTE TYPE ILW

Is the waste subject to

Scottish Policy:

No

WASTE VOLUMES

Comment on volumes: Final Dismantling & Site Clearance is assumed to commence in 2083 with reactor

dismantling commencing in 2087 and lasting for three years. Volumes and radioactivity

have been calculated for 85 years after reactor shutdown, i.e. 2087.

Uncertainty factors on volumes:

Stock (upper): x Arisings (upper) x 1.2 Stock (lower): x Arisings (lower) x 0.8

WASTE SOURCE Stainless steel items from reactor dismantling.

PHYSICAL CHARACTERISTICS

General description: A variety of stainless steel items. Waste can be packaged in standard ILW containers.

Physical components (%vol): Stainless steel items (100%).

Sealed sources: The waste does not contain sealed sources.

Bulk density (t/m³): ~1.4

Comment on density: The density is of the waste as cut for packaging.

CHEMICAL COMPOSITION

General description and components (%wt):

Stainless steels (100%).

Chemical state: Neutral

Chemical form of radionuclides:

C-14: The carbon 14 will be incorporated in the steel. There may also be some graphite

contamination.

CI-36: The chlorine-36 will be incorporated in the steel.

Tc-99: The chemical form of technetium has not been determined.

Metals and alloys (%wt): Items will have been cut for packaging. Thicknesses are likely to vary from a few mm to

about 25 mm.

(%wt) Type(s) / Grade(s) with proportions % of total C14 activity Stainless steel..... 100.0 Stainless steel types are BS980-100.0 CDS20, BS970-EN59, BS1631/1950 and BS970-EN58. Other ferrous metals..... Iron..... Aluminium...... 0 Beryllium...... 0 Greatest measured value from the Cobalt..... < 0.30 various components. Copper...... 0

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	Nickel			
	Titanium			
	Uranium			
	Zinc	0		
	Zircaloy/Zirconium	. 0		
	Other metals	TR	Silver and niobium	
Organics (%	Swt): None expected. No	halogena	ted plastics or rubbers will be present.	
		(%wt)	Type(s) and comment	% of total C14
	Total cellulosics	0		activity
	Paper, cotton	0		
	Wood	0		
	Halogenated plastics	0		
	Total non-halogenated plastics	0		
	Condensation polymers	0		
	Others	0		
	Organic ion exchange materials	0		
	Total rubber	0		
	Halogenated rubber	0		
	Non-halogenated rubber	0		
	Hydrocarbons			
	Oil or grease			
	Fuel			
	Asphalt/Tarmac (cont.coal tar)			
	Asphalt/Tarmac (no coal tar)			
	Bitumen			
	Others			
	Other organics	0		
Other mater	ials (%wt): Some graphite dus	st may be a	associated with reactor materials.	
		(%wt)	Type(s) and comment	% of total C14
		(70111)	Type(s) and comment	activity
	Inorganic ion exchange materials	0		
	Inorganic sludges and flocs	0		
	Soil	0		
	Brick/Stone/Rubble	0		
	Cementitious material	0		
	Sand			
	Glass/Ceramics	0		
	Graphite	TR		
	Desiccants/Catalysts			
	Asbestos	0		
	Non/low friable			
	Moderately friable			

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Highly friable		
Free aqueous liquids	0	
Free non-aqueous liquids	0	
Powder/Ash	0	
Inorganic anions (%wt): Trace quantities of	chloride m	ay be present.
	(%wt)	Type(s) and comment
Fluoride	0	
Chloride	TR	
lodide	0	
Cyanide	0	
Carbonate	0	
Nitrate	0	
Nitrite	0	
Phosphate	0	
Sulphate	0	
Sulphide	0	
Materials of interest for No materials likely waste acceptance criteria:	to pose a f	ire or other non-radiological hazard have been identified.
	(%wt)	Type(s) and comment
Combustible metals	0	
Low flash point liquids	0	
Explosive materials	0	
Phosphorus	0	
Hydrides	0	
Biological etc. materials	0	
Biodegradable materials		
Putrescible wastes	0	
Non-putrescible wastes		
Corrosive materials	0	
Pyrophoric materials	0	
Generating toxic gases	0	
Reacting with water	0	
Higher activity particles		
Soluble solids as bulk chemical compounds		
Hazardous substances / None expected non hazardous pollutants:		
	(%wt)	Type(s) and comment
Acrylamide		
Benzene		
Chlorinated solvents		
Formaldehyde		

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Organometallics			
Phenol			
Styrene			
Tri-butyl phosphate			
Other organophosphate	es		
Vinyl chloride			
Arsenic			
Barium			
Boron			
Boron (in Boral)			
Boron (non-Boral)			
Cadmium			
Caesium			
Selenium			
Chromium			
Molybdenum		<0.80	Greatest measured value from the various components.
Thallium			
Tin			
Vanadium			
Mercury compounds			
Others			
Electronic Electrical Ed	quipment (EEE)		
EEE Type 1			
EEE Type 2			
EEE Type 3			
EEE Type 4			
EEE Type 5			
Complexing agents (%wt): Yes			
		(%wt)	Type(s) and comment
EDTA			
DPTA			
NTA			
Polycarboxylic acids			
Other organic complexa	ants		
Total complexing agent	S	TR	
)/"substantial" thickness items considered nless items assumed DIs.
PACKAGING AND CONDITIONIN	NG		

PACK

The waste will be placed in baskets in the waste packages, and will be encapsulated in a BFS/OPC matrix. $\begin{tabular}{l} \end{tabular}$ Conditioning method:

Plant Name: None

Location: **Bradwell Decommissioning Site**

Plant startup date: 2087 **WASTE STREAM** 9B310 Stainless Steel (Reactor) ILW

Total capacity

~5000.0

(m³/y incoming waste):

Target start date for packaging this stream:

2087

~42.0

Throughput for this stream

(m³/y incoming waste):

The waste is not expected to be supercompacted. Baskets of different ILW wastes

may be in the same package.

Likely container

Other information:

type:

Container	Waste packaged (%vol)	Waste loading (m³)	Payload (m³)	Number of packages
4m box (200mm concrete shielding)	100.0	9.34	10.9	18

Likely container type

comment:

Container choice may be influenced by Transport Regulations at the time of final site clearance. The waste is assumed to be in baskets in the waste package so the occupied volume in the package is greater than the original waste volume. If the calculated average specific activity is exceeded it may be necessary to use containers with 300 mm of

shielding.

Range in container waste

volume:

Not yet determined. No significant variability is expected.

Other information on

containers:

The container material is expected to be stainless steel.

Likely conditioning matrix:

Other information:

Blast Furnace Slag / Ordinary Portland Cement The waste is assumed to be encapsulated.

~3.0 Conditioned density (t/m³):

Conditioned density

comment:

The conditioned waste density assumes that the waste will be encapsulated.

Other information on

conditioning:

The waste will be in baskets placed in the waste packages. Baskets of different Final Dismantling ILW wastes may be in the same waste packages. The encapsulation matrix

would be likely to be BFS/OPC.

Opportunities for alternative

disposal routing:

Baseline Management Route	Opportunity Management Route	Stream volume (%)	Estimated Date that Opportunity will be realised	Opportunity Confidence	Comment
_	_	_	_	_	_

RADIOACTIVITY

Source: Activation of the stainless steel and impurities.

The values quoted were derived by calculation from available material specification and Uncertainty:

are indicative of the activities that are expected. The major source of uncertainty is the

impurity levels.

Definition of total alpha and total beta/gamma:

Where totals are shown on the table of radionuclide activities they are the sums of the listed alpha or beta/gamma emitting radionuclides plus 'other alpha' or 'other beta/gamma'.

Measurement of radioactivities:

The specific activities were estimated from neutron activation calculations of the material

and its impurities.

Other information: The activities quoted are those at 85 years after reactor shutdown, i.e. in 2087. There may

be some contamination by Cs137.

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	Mean radioactivity, TBq/m³					Mean radioactivity, TBq/m³			
Nuclide	Waste at 1.4.2022	Bands and Code	Future arisings	Bands and Code	Nuclide	Waste at 1.4.2022	Bands and Code	Future arisings	Bands and Code
H 3				8	Gd 153	1			8
Be 10				8	Ho 163				8
C 14			2.75E-02	CC 2	Ho 166m				8
Na 22				8	Tm 170				8
Al 26				8	Tm 171				8
CI 36			9.84E-07	CC 2	Lu 174				8
			9.04L-07		Lu 174				8
Ar 39				8	Hf 178n				
Ar 42				8					8
K 40				8	Hf 182				8
Ca 41				8	Pt 193			0.045.00	8
Mn 53				8	TI 204			9.84E-08	CC 2
Mn 54				8	Pb 205				8
Fe 55			2.21E-08	CC 2	Pb 210				8
Co 60			1.5E-04	CC 2	Bi 208				8
Ni 59			5.4E-02	CC 2	Bi 210m				8
Ni 63			3.58E+00	CC 2	Po 210				8
Zn 65				8	Ra 223				8
Se 79				8	Ra 225				8
Kr 81				8	Ra 226				8
Kr 85				8	Ra 228				8
Rb 87				8	Ac 227				8
Sr 90				8	Th 227				8
					Th 228				8
Zr 93				8	Th 229				8
Nb 91			4 005 00	8					
Nb 92			1.38E-09	CC 2	Th 230				8
Nb 93m				6	Th 232				8
Nb 94			8.46E-04	CC 2	Th 234				8
Mo 93			9.83E-05	CC 2	Pa 231				8
Tc 97				8	Pa 233				8
Tc 99			1.86E-05	CC 2	U 232				8
Ru 106				8	U 233				8
Pd 107				8	U 234				8
Ag 108m			1.09E-04	CC 2	U 235				8
Ag 110m				8	U 236				8
Cd 109				8	U 238				8
Cd 113m				8	Np 237				8
Sn 119m				8	Pu 236				8
Sn 121m				8	Pu 238				8
Sn 121111				8	Pu 239				8
					Pu 240				8
Sn 126				8	Pu 241				8
Sb 125				8	Pu 241				8
Sb 126				8	Am 241				
Te 125m				8		1			8
Te 127m				8	Am 242m				8
l 129				8	Am 243				8
Cs 134				8	Cm 242				8
Cs 135				8	Cm 243				8
Cs 137				6	Cm 244				8
Ba 133				8	Cm 245				8
La 137				8	Cm 246				8
La 138				8	Cm 248				8
Ce 144				8	Cf 249				8
Pm 145				8	Cf 250				8
Pm 147				8	Cf 251				8
Sm 147				8	Cf 252				8
					Other a				Č
Sm 151				8	Other b/g				
Eu 152	1			8	Total a	0		0	
Eu 154									
Eu 155				8 8	Total b/g	o o		3.66E+00	CC 2

Bands (Upper and Lower)

A a factor of 1.5 B a factor of 3 C a factor of 10

D a factor of 100 E a factor of 1000

Note: Bands quantify uncertainty in mean radioactivity.

Code

- Measured activity
 Derived activity (best estimate)
 Derived activity (upper limit)
- 4 Not present
- 5 Present but not significant
- 6 Likely to be present but not assessed
- 7 Present in significant quantities but not determined
- 8 Not expected to be present in significant quantity